



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
REGION II  
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200  
ATLANTA, GEORGIA 30303-1200

March 26, 2025

Eric S. Carr  
President – Nuclear Operations  
and Chief Nuclear Officer  
Innsbrook Technical Center  
5000 Dominion Blvd.  
Glen Allen, VA 23060-6711

SUBJECT: NOTIFICATION OF NORTH ANNA POWER STATION UNITS 1 & 2  
COMPREHENSIVE ENGINEERING TEAM INSPECTION - U.S. NUCLEAR  
REGULATORY COMMISSION INSPECTION REPORT 05000338/2025010 AND  
05000339/2025010

Dear Eric S. Carr:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a Comprehensive Engineering Team Inspection (CETI) at your North Anna Power Station, Units 1 & 2, during the weeks of July 14 and July 28, 2025. Geoffrey Ottenberg, a Senior Reactor Inspector from the NRC's Region II office, will lead the inspection team. The inspection will be conducted in accordance with Inspection Procedure 71111.21M, "Comprehensive Engineering Team Inspection (CETI)," issued October 7, 2022.

The inspection will evaluate the capability of components that have been modified and risk-significant/low-margin components to function as designed and to support proper system operation. The inspection will also include a review of selected operator actions, and operating experience, and modifications.

We confirmed arrangements with . Bob Page for an information-gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information-gathering visit: Week of June 2, 2025
- Onsite weeks: Weeks of July 14 and July 28, 2025

The purpose of the information-gathering visit is to meet with members of your staff to identify components that have been modified, risk-significant components, and operator actions. Information and documentation needed to support the inspection will also be identified. Andy Rosebrook, a Region II Senior Reactor Analyst, will support Geoffrey. Ottenberg during the information-gathering visit to review probabilistic risk assessment data and identify components to be examined during the inspection. Additionally, during the onsite weeks, time may be needed on the plant-referenced simulator to facilitate the development of operator action-based scenarios.

The enclosure lists documents that will be needed prior to the information-gathering visit. Please provide the referenced information to the Region II Office by Monday, May 26, 2025. Additional documents will be requested following the information-gathering visit.

The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation. The additional information will be needed in the Region II office by Friday, July 4, 2025, to support the inspection team's preparation week. During the information-gathering trip, Mr. Ottenberg will also discuss the following inspection support administrative details: (1) availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection; (2) method of tracking inspector requests during the inspection; (3) licensee computer access; (4) working space; (5) arrangements for site access; and (6) other applicable information.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with 10 CFR 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Thank you for your cooperation in this matter. If you have any questions, regarding the information requested or the inspection, please contact Mr. Ottenberg at 404-997-4658 or contact me at 404-997-4630.

Sincerely,



Signed by Taylor, Ryan  
on 03/26/25

Ryan C. Taylor, Chief  
Engineering Branch 1  
Division of Operating Reactor Safety

Docket Nos. 05000338 and 05000339  
License Nos. NPF-4 and NPF-7

Enclosure:  
Information Request for North Anna Power Station Units 1 & 2 Comprehensive Engineering  
Team Inspection

cc w/ encl: Distribution via LISTSERV

## INFORMATION REQUEST FOR NOTH ANNA POWER STATION UNITS 1 & 2 COMPREHENSIVE ENGINEERING TEAM INSPECTION

Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM (or FTP site, SharePoint, etc.). The CDROM (or website) should be indexed and hyperlinked to facilitate ease of use. The requested items below, identified with an asterisk (\*), should have a date range from **January 1, 2022, until present**.

1. \*List of modifications to systems, structures or components (SSCs) that are **permanent and field work completed**. (For the purpose of this inspection, modifications include permanent: plant changes, design changes, calculation changes, procedure changes, and set point changes.)

The list should contain the number of the modification, a short description, affected system and unit. If multiple categories/stages of modifications exist, provide a description of the types relative to the coding scheme in your transmittal. Additionally, the list should be categorized by quality level (i.e., safety or non-safety) as well as whether accompanied by a screening or 10 CFR 50.59 evaluation. A query report return-field-set might look like:

Identifier	Unit	System <sup>1</sup>	Description	Type <sup>2</sup>	Status <sup>3</sup>	QA Condition <sup>4</sup>	50.59 <sup>5</sup>
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- <sup>1</sup> e.g. If systems are provided with some type of number or acronym designation, define the system's designation on a separate list or document
- <sup>2</sup> e.g. Engineering Design/Document Change, Design Change Notice, Test, Experiment, etc.
- <sup>3</sup> e.g. Closed, Returned to Operation, Void, Partially Completed, Released for Installation, Phase 1 Completed, etc.
- <sup>4</sup> e.g. Safety Related or Non-Safety Related
- <sup>5</sup> e.g. Screened Out or Full 10 CFR 50.59 Evaluation

2. From your most recent probabilistic safety analysis (PSA) excluding external events and fires:

- a. Two risk rankings of components from your site-specific PSA: one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance

- b. A list of the top 500 cut-sets

- c. A list of the top 500 LERF contributors

3. From your most recent PSA including external events and fires:

- a. Two risk rankings of components from your site-specific PSA: one sorted by RAW, and the other sorted by Birnbaum Importance

- b. A list of the top 500 cut-sets

4. Risk ranking of operator actions from your site-specific PSA sorted by RAW and human reliability worksheets for these items

5. List of time-critical operator actions with a brief description of each action with applicable administrative procedures used to maintain them

6. \*List of components with low-design margins (i.e., pumps closest to the design limit for flow or pressure, diesel generator close to design-required output, heat exchangers close to

Enclosure

rated design heat removal, and motor-operated valve risk-margin rankings, etc.) and associated evaluations or calculations

7. \*List and brief description of Root Cause Evaluations performed
8. \*List and brief description of common-cause component failures that have occurred
9. List and brief description of equipment currently in degraded or nonconforming status as described in NRC Generic Letter 91-18, Revision 1
10. \*List and brief description of Operability Determinations and Functionality Assessments
11. \*List and reason for equipment that has been classified in maintenance rule (a)(1) status
12. \*List of equipment on the site's Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list, and summaries (if available) of your plans to address the issue(s) along with dates added or removed from the issues list
13. List of current "operator work arounds/burdens"
14. Copy of Technical Specifications Bases
15. Copy of Technical Requirements Manual (TRM)
16. Copy of the Quality Assurance Program Description document
17. Copy of Corrective Action Program Procedure(s)
18. Copy of Operability Determination Procedure(s)
19. Copy of procedures addressing the following: Modifications, design changes, set point changes, equivalency evaluations or suitability analyses, commercial grade dedications, post-modifications testing, 10 CFR 50.59 screenings and evaluations, and UFSAR updates
20. Copy of procedures addressing the following: loss of service water system or ultimate heat sink (UHS); controls to prevent clogging due to macrofouling; and performance, testing, and inspection requirements for service water system and UHS
21. List of heat exchangers or equipment cooled by service water directly or indirectly. Include the risk ranking for each listed heat exchanger.
22. List of motor operated valves and air operated valves in valve programs, and their associated design margin and risk ranking
23. Primary AC and DC calculations for safety-related buses
24. One-line diagram of electrical plant (Electronic only)
25. Index and legend for electrical plant one-line diagrams
26. Piping and instrumentation diagrams (P&IDs) for safety-related systems (Electronic)
27. Index and legend for P&IDs
28. Index (procedure number, title, and current revision) of station Emergency Operating Procedures, Abnormal Operating Procedures, and Annunciator Response Procedures

29. Copies of corrective action documents generated from previous design bases assurance inspections
30. Copy of any self-assessments performed, and corrective action documents generated, in preparation for current comprehensive engineering team inspection
31. Contact information for a person to discuss PSA information prior to and during the information-gathering trip (Name, title, phone number, and e-mail address)
32. List of system acronyms